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# **Operational Reactor Safety**

## **22.091/22.903**

Professor Andrew C. Kadak  
Professor of the Practice

## Boiling Water Reactors

### Lecture 15



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# Topics to be Covered

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- Steam Cycle
  - Recirculation
  - Chimney
  - Steam separation
  - Heat removal
  - Operating with voids
  - Plant systems
  - Reactor Protective System
  - Safety Systems
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# BWR Plant Layout

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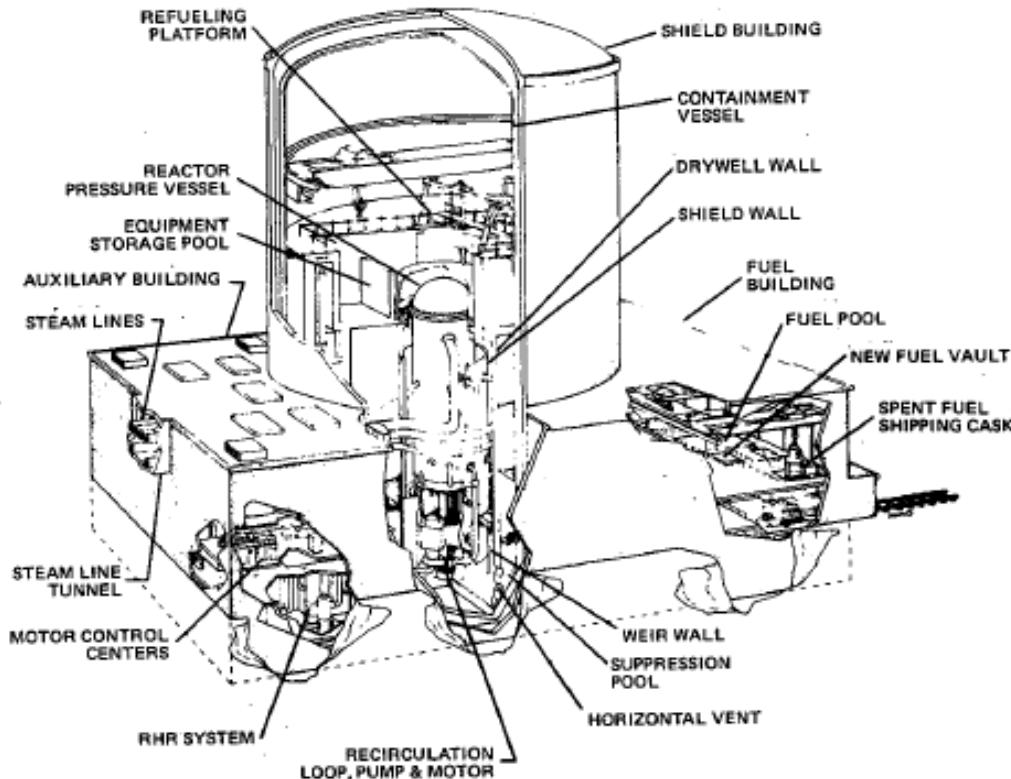


Figure 7-1. Reactor Building, Fuel Building, and Auxiliary Building

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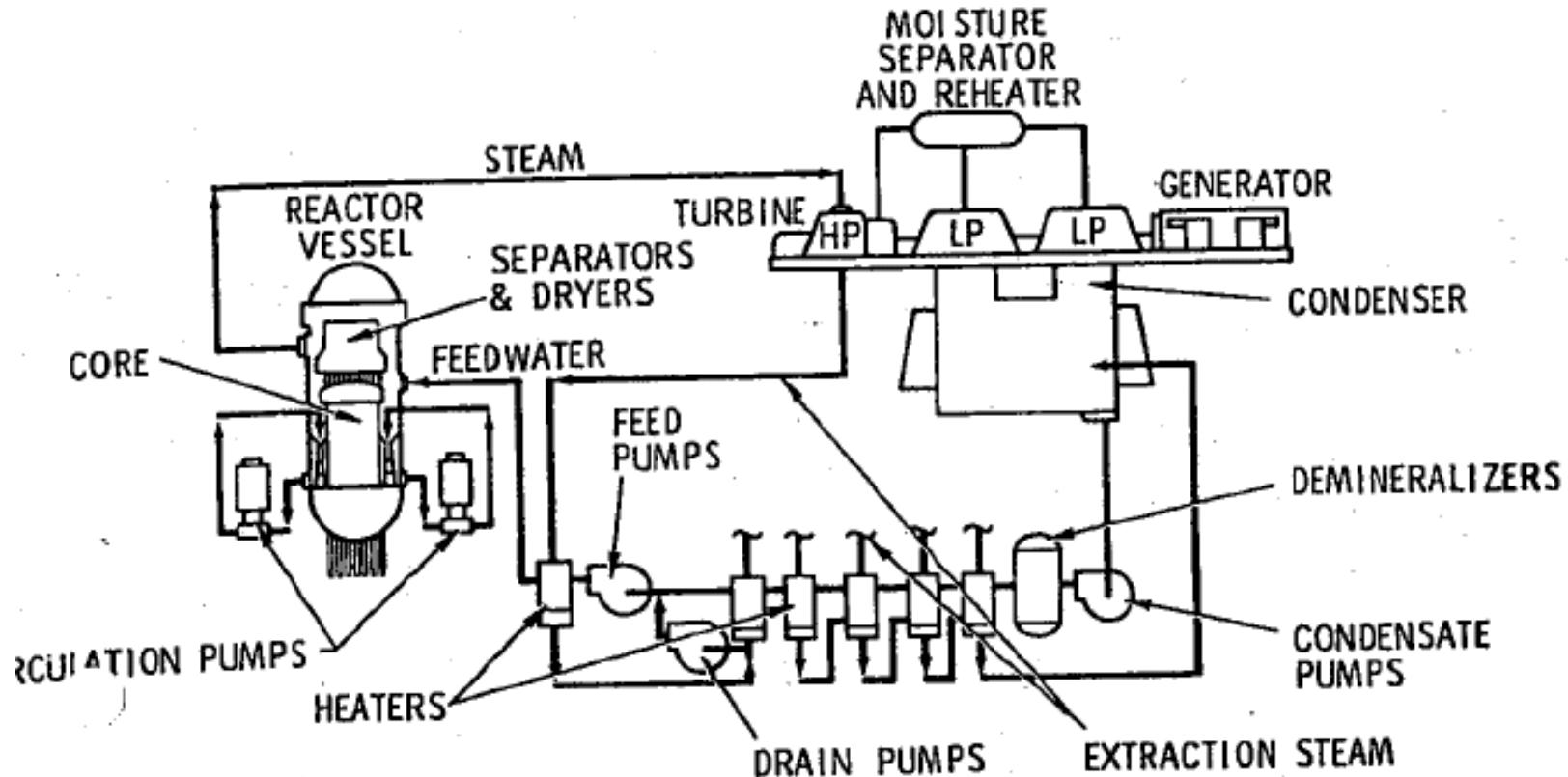
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# BWR Plant Schematic

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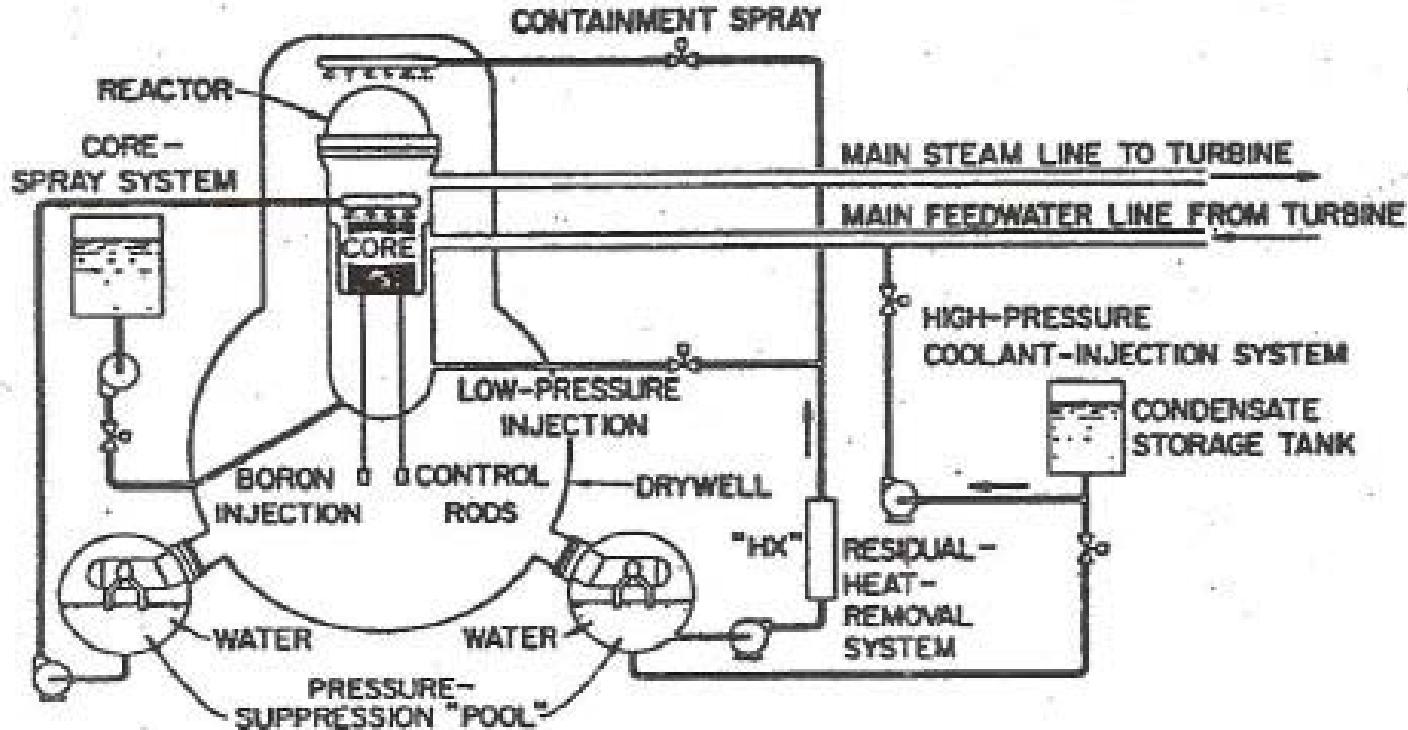


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# BWR Early



**FIGURE 14-6**

Engineered safety systems for an early BWR. (From W. B. Cottrell, "The ECCS Rule-Making Hearing," *Nuclear Safety*, vol. 15, no. 1, Jan.-Feb. 1974.)

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# Reactor Assembly

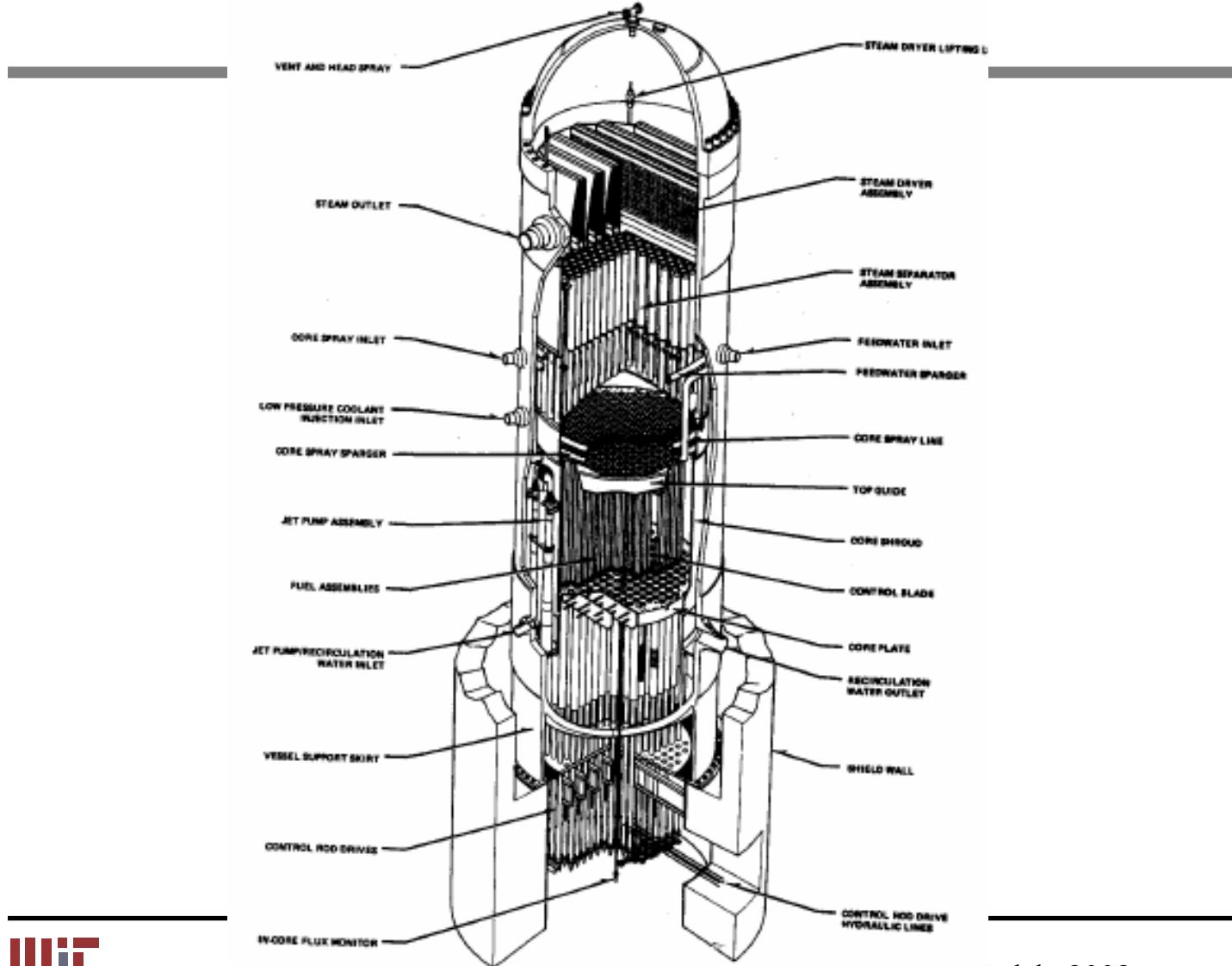
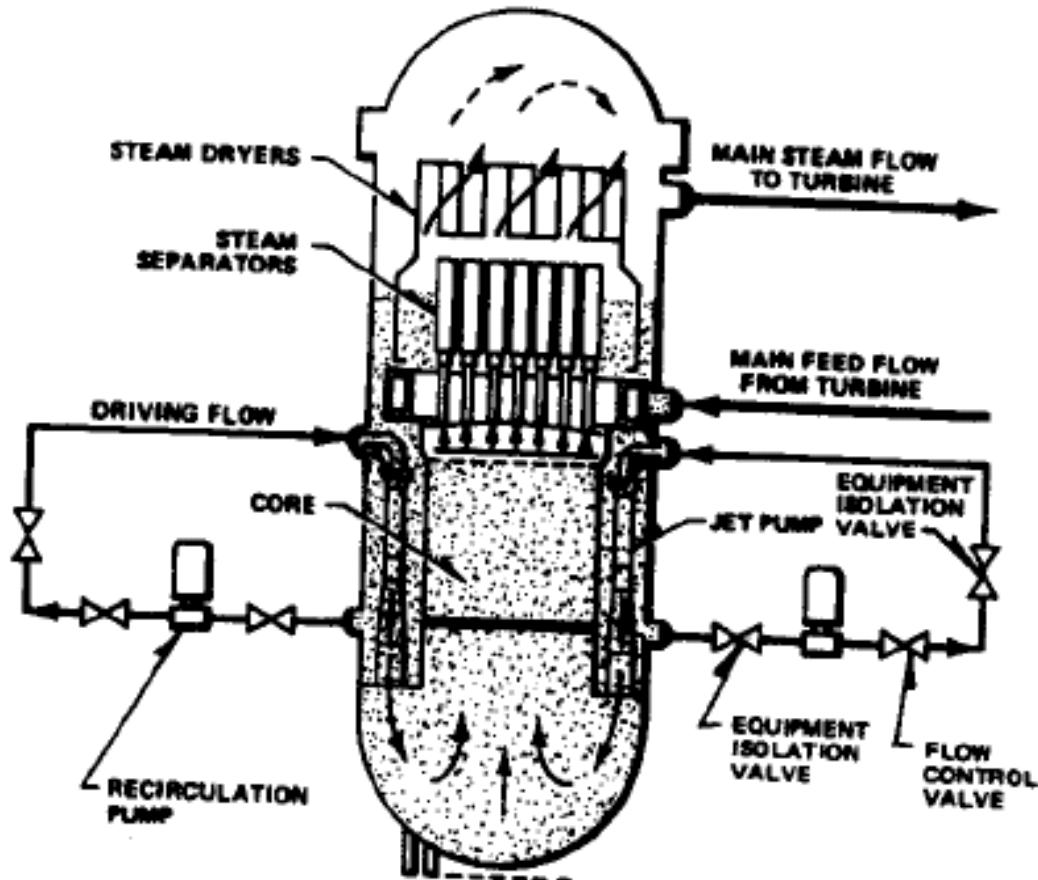


Figure 2-1. Reactor Assembly

# Steam and Recirculation System

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# Jet Pumps – Recirculation System

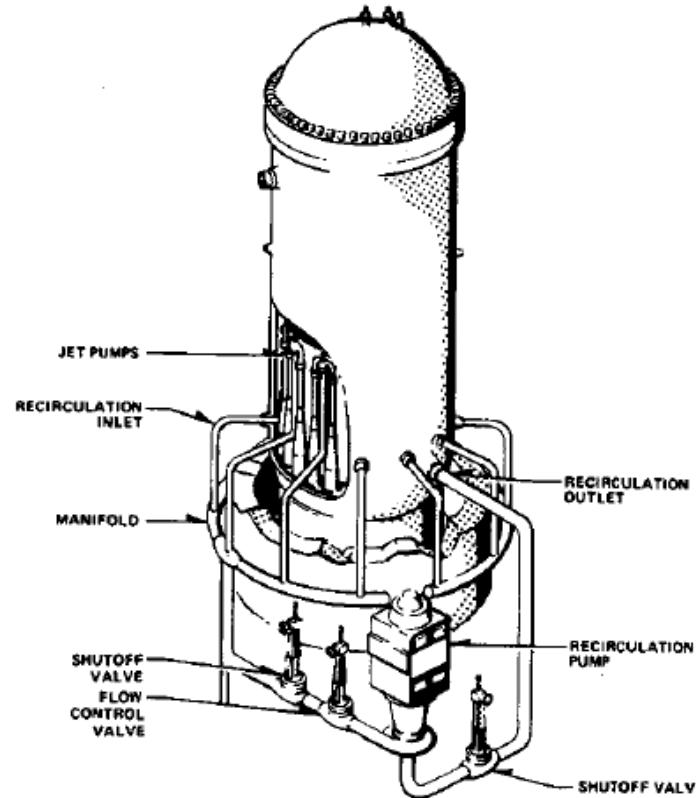
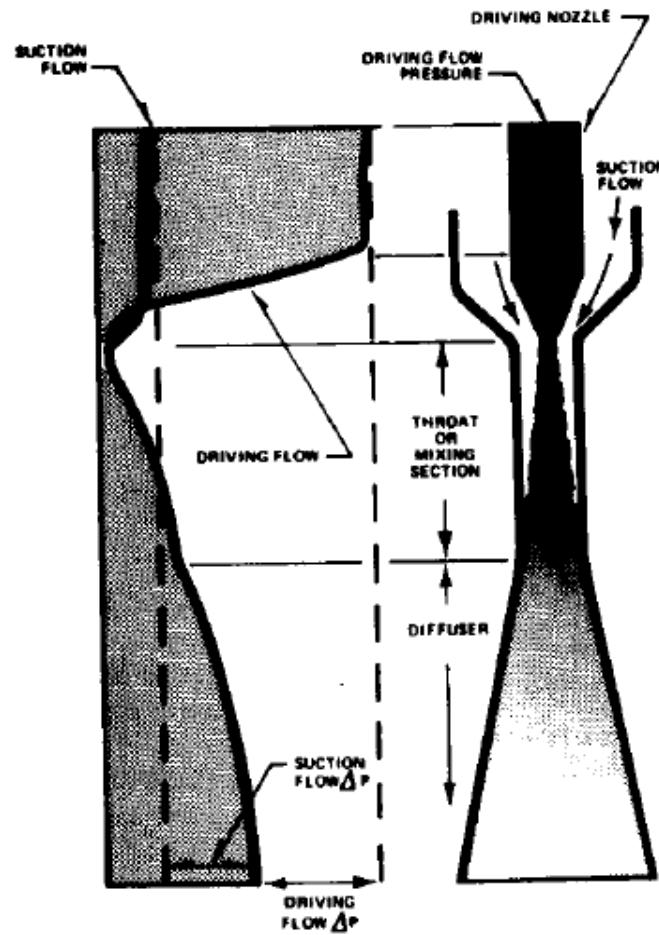


Figure 2-5. BWR Vessel Arrangement for Jet Pump Recirculation System



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# Steam Separator and Dryer

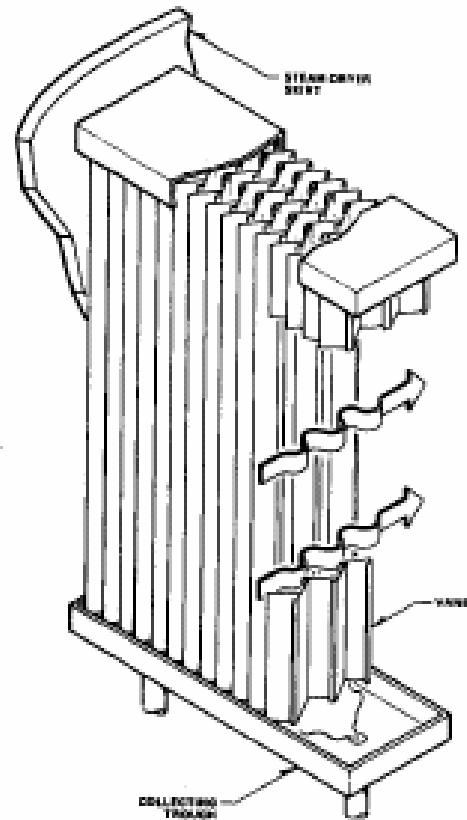
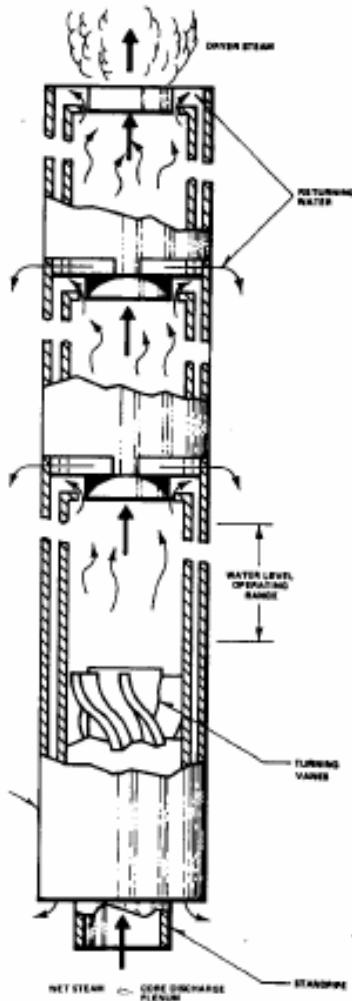


Figure 2-3. Steam Dryer

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# Key Systems of BWRs

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- Standby Liquid Control System (SLCS)
    - For redundant shutdown – contains boron
  - Reactor Core Isolation Cooling System (RCIC)
    - Deals with loss of feedwater flow
  - Emergency Core Cooling System (ECCS)
    - Safety Relief Valve – Automatic Depressurization System
    - High Pressure Coolant Injection System (HPCI)
    - Low Pressure Coolant Injection System (LPCI)
  - Control Rod Drive System
    - Hydraulic Control Units – from bottom of reactor vessel
  - Residual Heat Removal System – decay heat removal
- 



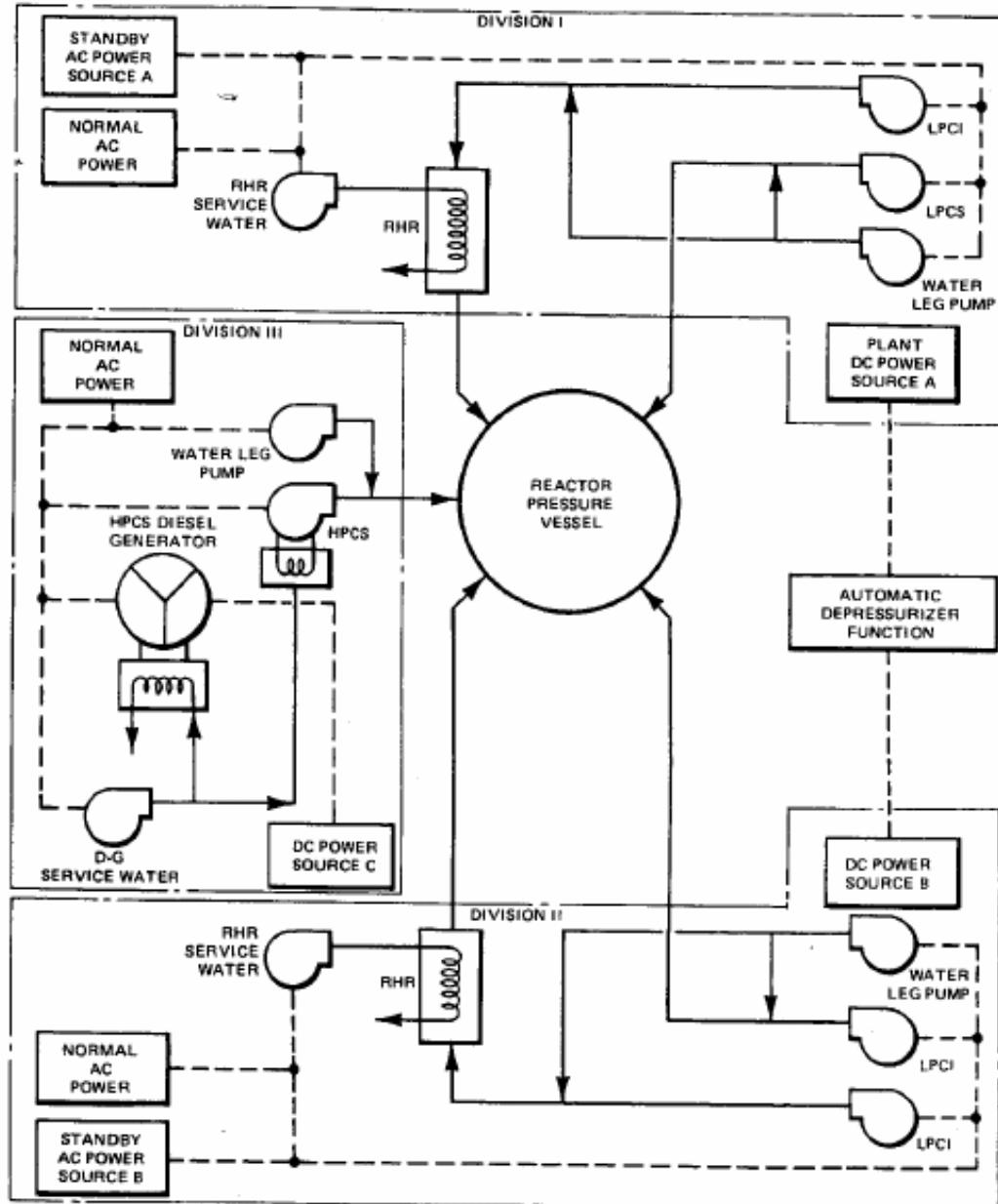
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# ECCS



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Figure 4-7. Emergency Cooling System Network

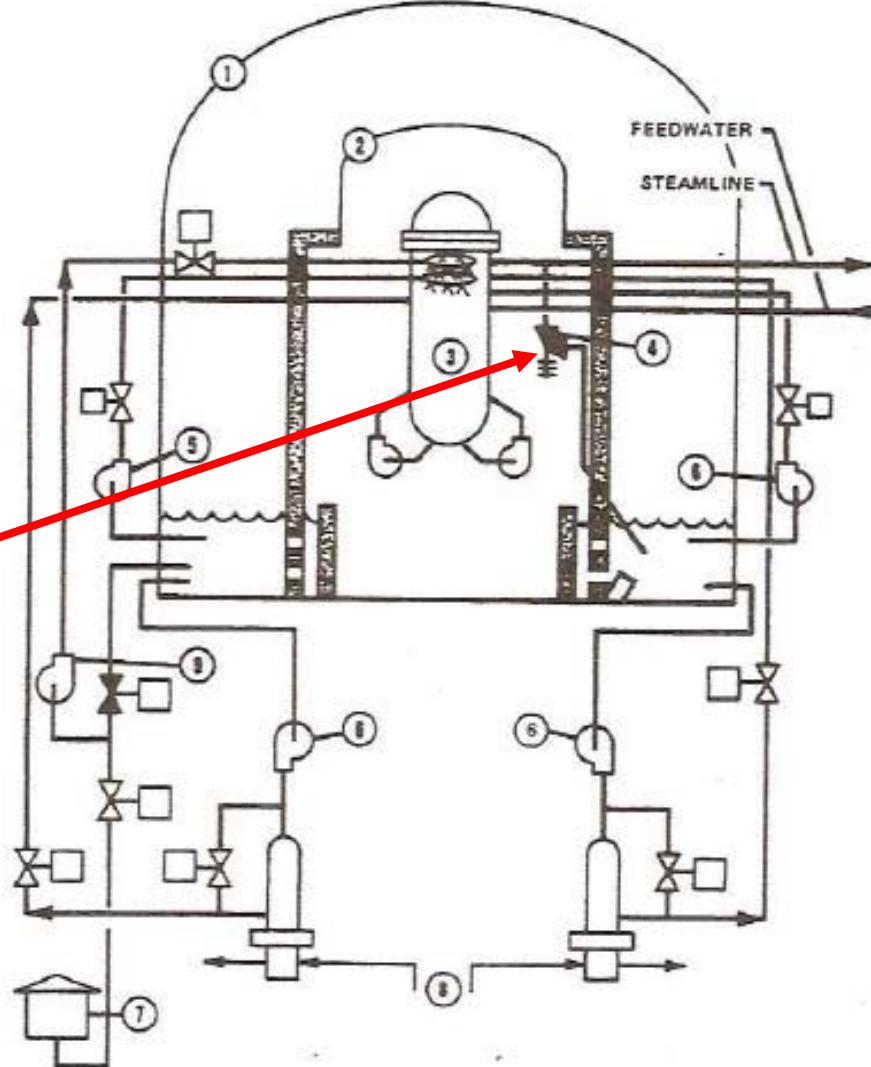
RHR – RESIDUAL HEAT REMOVAL  
HPCS – HIGH PRESSURE CORE SPRAY

LPCS – LOW PRESSURE CORE SPRAY  
LPCI – LOW PRESSURE COOLANT INJECTION MODE OF RHR

— ELECTRICAL  
— PIPING

# ADS

ADS Valve



1. CONTAINMENT
2. DRYWELL
3. RPV
4. SAFETY/RELIEF VALVE DEPRESSURIZATION
5. LOW PRESSURE SPRAY
6. LOW PRESSURE COOLING INJECTION FUNCTION OF RESIDUAL HEAT REMOVAL SYSTEM
7. CONDENSATE STORAGE TANK
8. SERVICE WATER
9. HIGH PRESSURE CORE SPRAY



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Figure 4-6. Emergency Core Cooling System

# Reactor Power Control

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- Control rods – gross power changes
  - Recirculation Flow - +/- 25 % power
    - Increase flow – increase power
  - Turbine Control – Pressure control - constant
    - Generator demands more power – turbine slows down – pressure decreases – more steam created - increase turbine power –then increase recirculation flow to compensate for reactivity loss.
  - Recall BWRs have negative void coefficient and Positive pressure coefficient
- 



# Reactor Trip Systems

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- High Pressure in Drywell
  - Low water level in reactor vessel
  - High pressure in reactor vessel
  - High neutron flux
  - High water level in scram discharge volume (control rods)
  - Closing of turbine stop or fast closure of turbine control valves
  - Main steam line isolation
  - High radiation levels in main steam lines
  - Leak detection
  - Low pressure in turbine inlet
- 



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# Trip Functions

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- Insertion of control rods (hydraulic control units in bottom)
- Nuclear System Isolation
  - Reactor Coolant Pressure Boundary
  - Containment Isolation
  - Closed System Isolation



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# Homework

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- Read BWR handouts
- Problem 3.3 El Wakil



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