

# 22.106 Neutron Interactions and Applications

## Problem Set 3

Due SES #12

### Question 1

Shielding: Monte Carlo simulation

1	2	3	4
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#### Problem Description

Slab 1: Source

- Source of Cf-252 producing neutrons of  $10^8$  neutrons per second uniformly distributed in the slab.
- Assume that all neutrons are produced at 1 MeV
- Width of 1 cm

Slab 2: Moderator

- Material is borated concrete
- Width of 5 cm

Slab 3: Gamma shield

- Material is lead
- Width of 2 cm

Slab 4: Person

- Width of 3 cm

Density of lead 11 g/cc

Density of concrete 2.5 g/cc

Density of the californium 15g/cc

Total cross section of lead: 4.39 b  
Absorption cross section of lead: 0.0033 b  
(n,2n) cross section of lead: 0.09 b  
Assume lead is a pure component of Pb-208.

Total cross section of borated concrete: 3.50 b  
Absorption cross section of borated concrete (if  $E > 0.1$  MeV): 0.01 b  
Absorption cross section of borated concrete (if  $E < 0.1$  MeV): 1.5 b  
Assume that the molecular mass of concrete is 60 g/mol and is made of a single component.

Total cross section of Californium-252: 0.1 b  
Absorption cross section of Californium-252: 0.001 b

Total macroscopic cross section of person:  $1 \text{ cm}^{-1}$   
Absorption cross section of person:  $0.2 \text{ cm}^{-1}$

The system is surrounded by a vacuum. Assume that the remainder of all total cross sections is elastic scattering and that all scattering is isotropic in the lab system.

### Questions

- 1) Using Analog Monte Carlo, compute the flux and absorption rate in the person (slab 4) and include the uncertainties.
- 2) Compare the figure of merit if you were to use:
  - Implicit Capture
  - Source Biasing
  - Implicit Capture and Source Biasing

### **Question 2**

Explain in your own words the basic idea of variance reduction and discuss its importance in Monte Carlo simulations.

### **Question 3**

Describe how the lack of information of potential  $V$  is circumvented in the R-matrix derivation.

### **Question 4**

Explain in your own words the roles of experimental data and theoretical modeling in evaluating nuclear cross sections. Why do we need to evaluate data?

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